

Simulation of Radionuclide Transport in Clay

Seminar presentation

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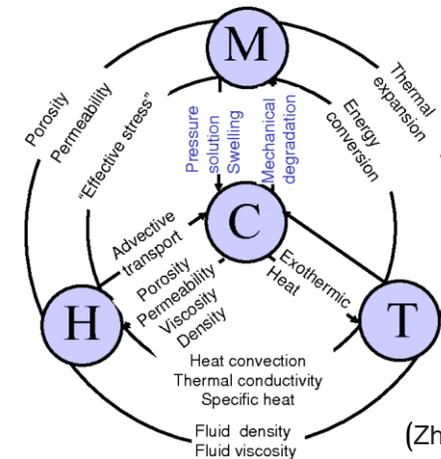
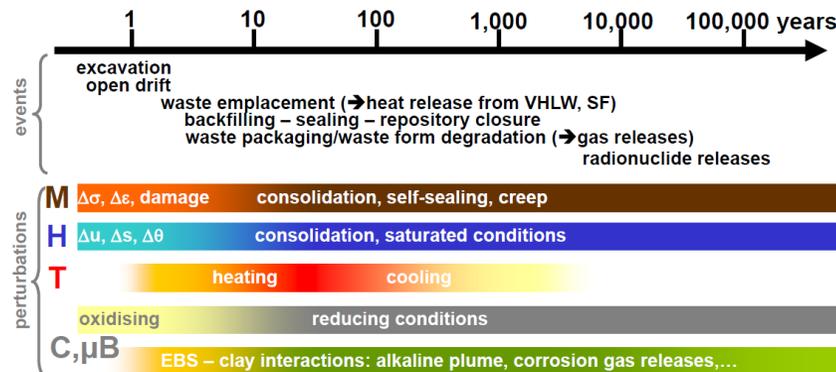
DECOVALEX

<https://decovallex.org/D-2027>

DEvelopment of COupled models and their VALidation against EXperiments

- DECOVALEX was initiated in 1992 as an international cooperative project to address modeling challenges in **deep geological repository (DGR)** systems, involving nuclear waste organizations, regulators, and research teams.
- DECOVALEX focuses on advancing **multiphysics simulations**, improving **coupled processes** (e.g. thermo(T), hydraulic(H), mechanical(M), chemical(C)) models, validating numerical models through **laboratory and field experiments**, and provide a platform for knowledge exchange and capacity building.

Major T, H, M, and C changes in both engineering and natural barriers during the lifetime of a geologic repository.



Selected coupling effects between T, H, M, and C processes

(Zheng et al., 2011)

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Outline

Introduction

- HLW final disposal
- KBS-3 disposal concept

Modeling Object

- Iodine-129
- Cigar Lake uranium deposit

Current Results

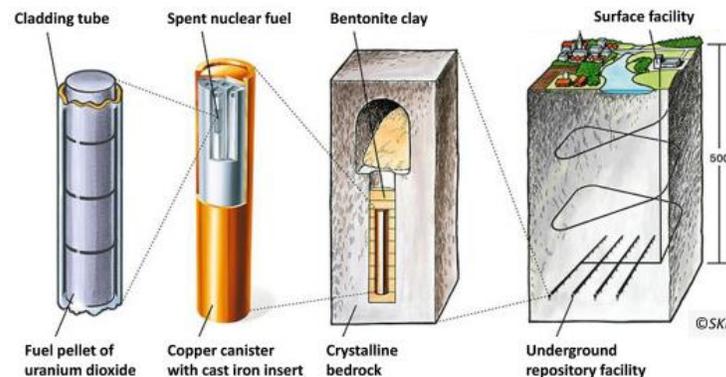
Conclusion & Future work

Introduction



- **High-level radioactive waste (HLW)**, comprising spent nuclear fuel (SNF) and reprocessed waste, is **highly radioactive**, has **long half-lives**, and **poses significant toxicity**, necessitating safe disposal and long-term isolation.
- **Deep geological disposal** with **engineering barrier** is generally adopted worldwide for **final disposal of HLW** management.
- KBS-3 disposal concept (SKB, 1983)
 - **multi-barrier system** = **engineering barrier** + **natural barrier**

Engineering barrier system:
Canister
Buffer
Backfill



Natural barrier system:
Host rock

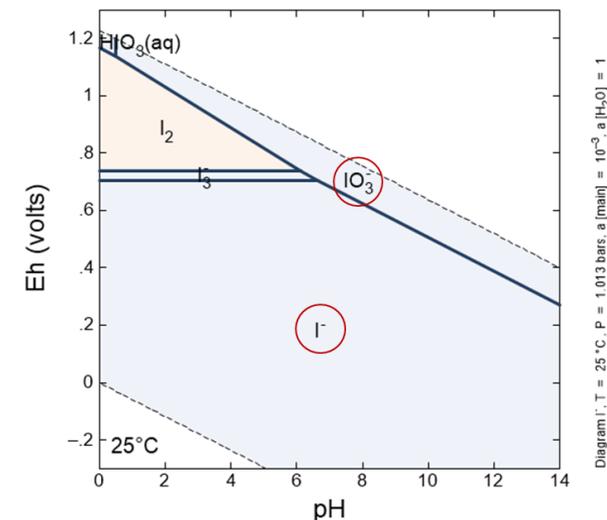
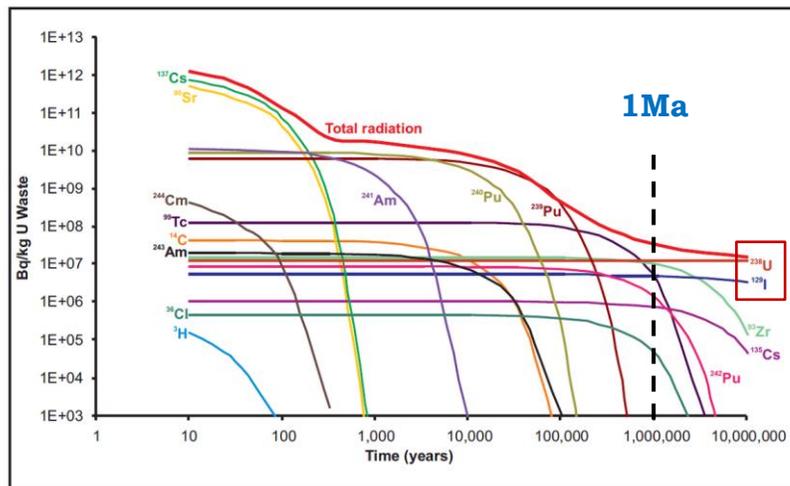
(SKB, 2006)

Modeling Object - Iodine-129

- Iodine-129 (I-129) is primarily produced through uranium fission. In the HLW safety assessment, I-129 is a significant radionuclide.
 - **Long half-life** (15.7Ma): It remains radioactive for an extremely long period.
 - **High mobility**: In aqueous environments, I-129 primarily exists as iodide (I^-) and iodate (IO_3^-), both of which are highly mobile.
 - **Migration modeling and long-term safety analysis** are necessary to predict and evaluate its potential effects on the environment and human health.

Dose contribution of major fission products in SNF. The bold red line is the total radiation.

https://commons.wikimedia.org/wiki/File:Radioactivity_of_Fission_Products.png



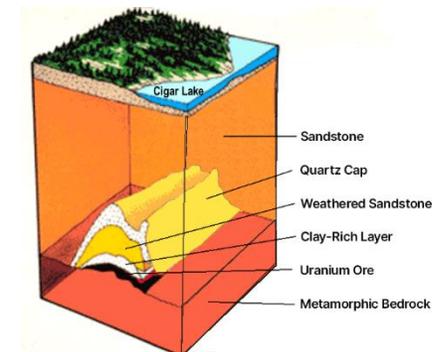
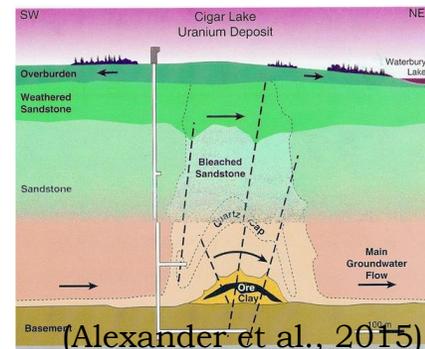
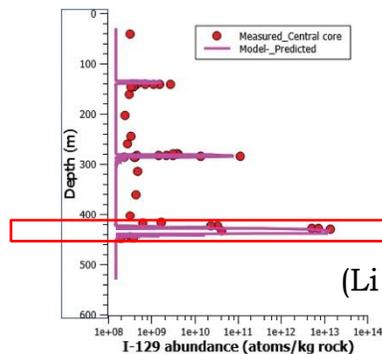
Objectives

- To verify the feasibility of the model by comparing simulation results with field data.
- To understand the long-term transport behavior of I-129 in fractured and porous media.
- To calibrate long-term simulation predictions and contribute to the HLW final disposal.

1 dimension

2 dimension

3 dimension

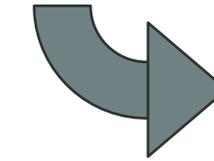
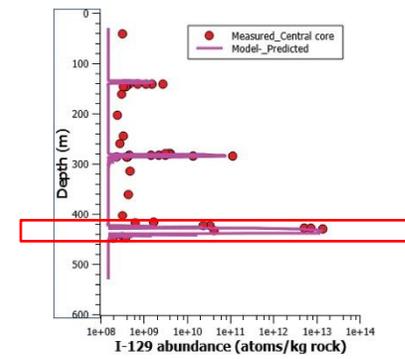




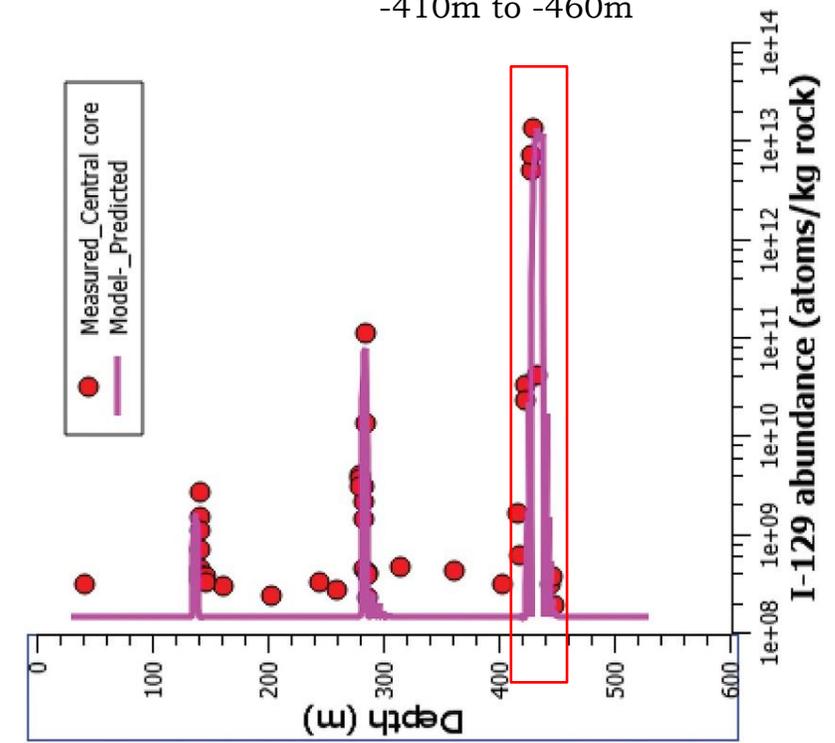
Current Result

Model Setting

- Simulation focuses on a depth range from -410 meters to -460 meters.
- The model is in the range from **-25m to 25m**, with center of uranium orebody located at 0.
- The material is **homogeneous**.
- **Diffusion-only** study case.



-410m to -460m



(Li et al., 2024)

Model Setting

- I-129 is primarily produced through **uranium** fission.
- Initial concentration of
 - I-129 is set as **0**.
 - U-238 follows a normal **gaussian** distribution.
- Dirichlet boundary condition
 - Concentration of I-129 on right and left **ends are set as 0**.
- Simulation time
 - The model can be run for up to **1.4 billion years**, which is the age of the uranium formation.

Governing Equation

- Diffusion equation (1D)

$$\varepsilon \frac{\partial c}{\partial t} + k_d \rho \frac{\partial c}{\partial t} = D_e \frac{\partial^2 c}{\partial x^2}$$

$$R = 1 + \frac{k_d \rho}{\varepsilon}$$

retardation factor

$$\frac{\partial c}{\partial t} = \frac{D_e}{R} \frac{\partial^2 c}{\partial x^2}$$



- Source term
- I-129 Production and Decay

$$S = \alpha[U]Y_f\lambda_{sf} - \lambda_d * c$$

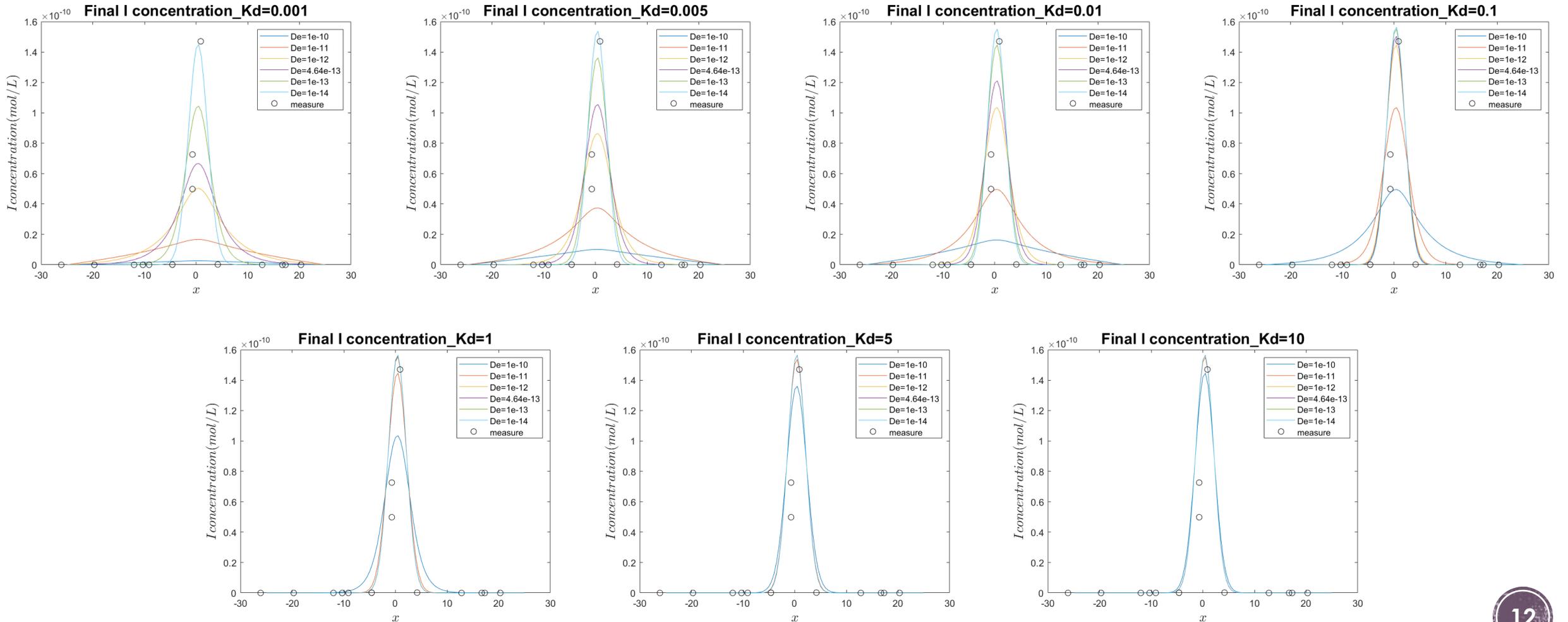
Model Parameters

concentration	c	mol/L
location	x	m
porosity	ε	-
Diffusion coefficient	D_e	m^2/s
Distribution coefficient of I	K_d	m^3/kg
Density	ρ	kg/m^3

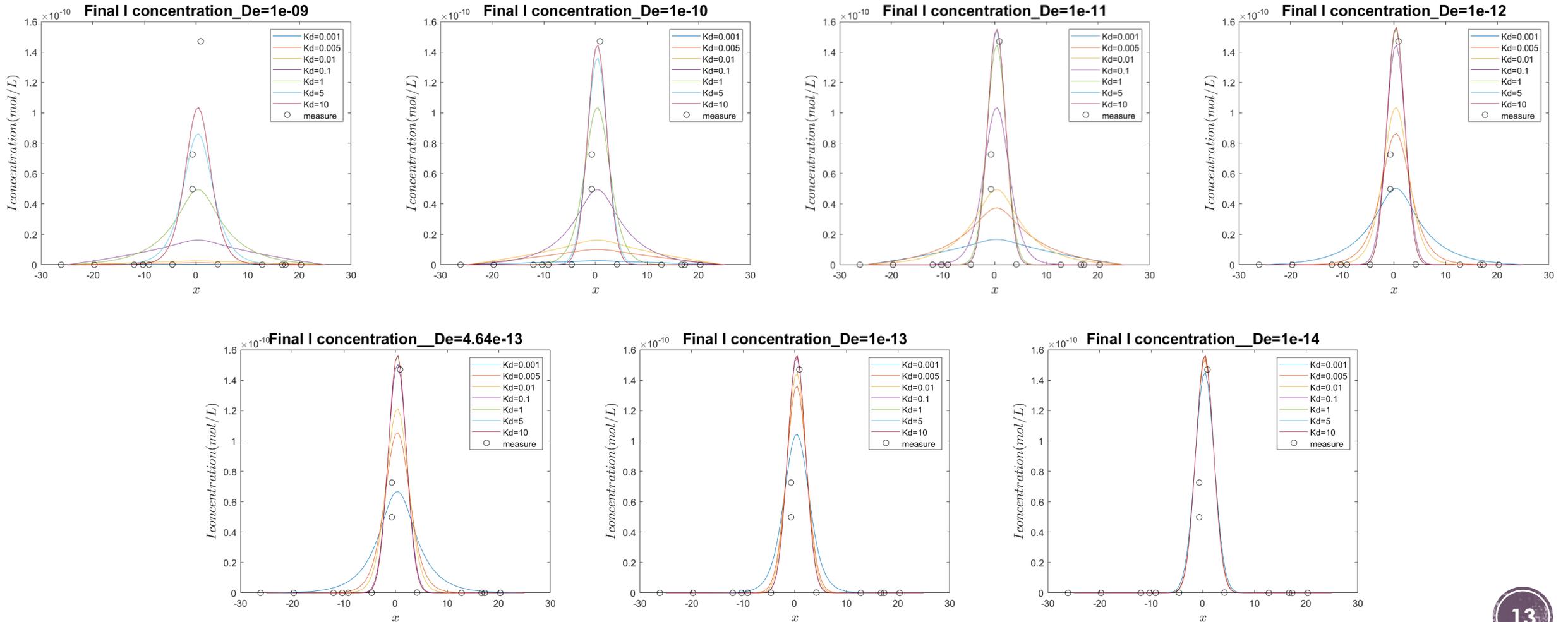
Model Parameters

empirical adjustment factor for I-129	α
fission yield at mass I-129 from the spontaneous fission (half life 16.1 Ma)	Y_f
decay constant for spontaneous fission of U-238 (half life 8.4e15 a)	λ_{sf}
rate of loss of I-129	λ_d
concentration	c

Results

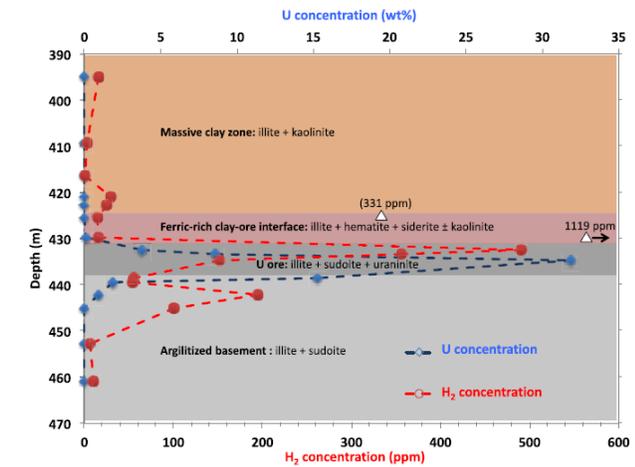


Results



Conclusion and Future Work

- The simulation results do not fit well with the measured data. One possible explanation for this might be the assumption of homogeneity in the model. In real case, the geological medium likely exhibits **heterogeneity**, including variations in mineral composition and porosity.
- The constant D_e and K_d values used in the model may oversimplify the system. Factors such as **chemical reaction**, scale effects and localized fractures could alter effective diffusion coefficients and adsorption behavior, influencing radionuclide transport.
- Future work should include **heterogeneous conditions**, through assigning distinct properties to different regions. Additionally, transitioning to **two-dimensional** and **three-dimensional** simulations could provide a more accurate representation of complex transport dynamics in natural systems.



(Truche et al., 2018)

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Thank you for your attention!

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